

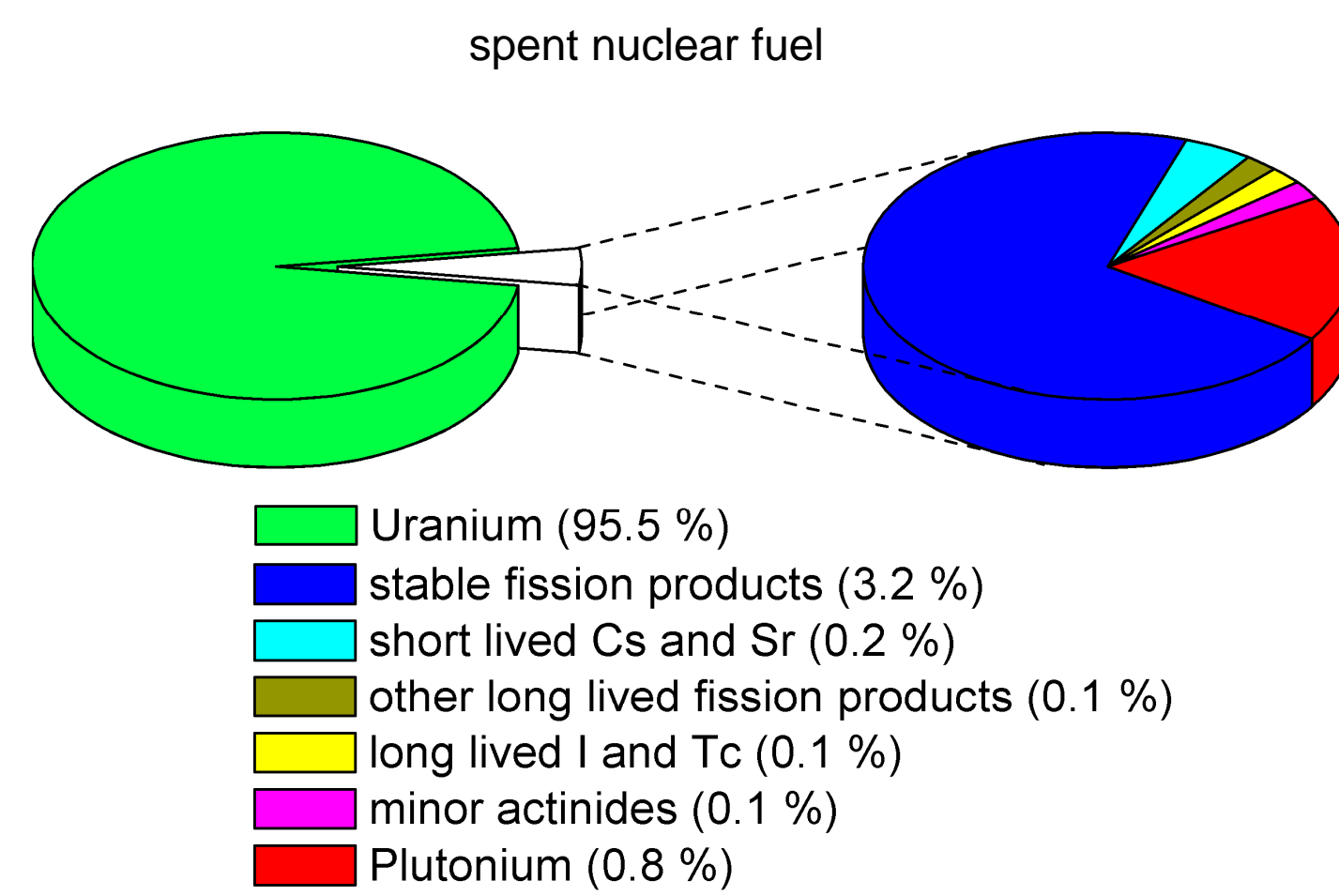
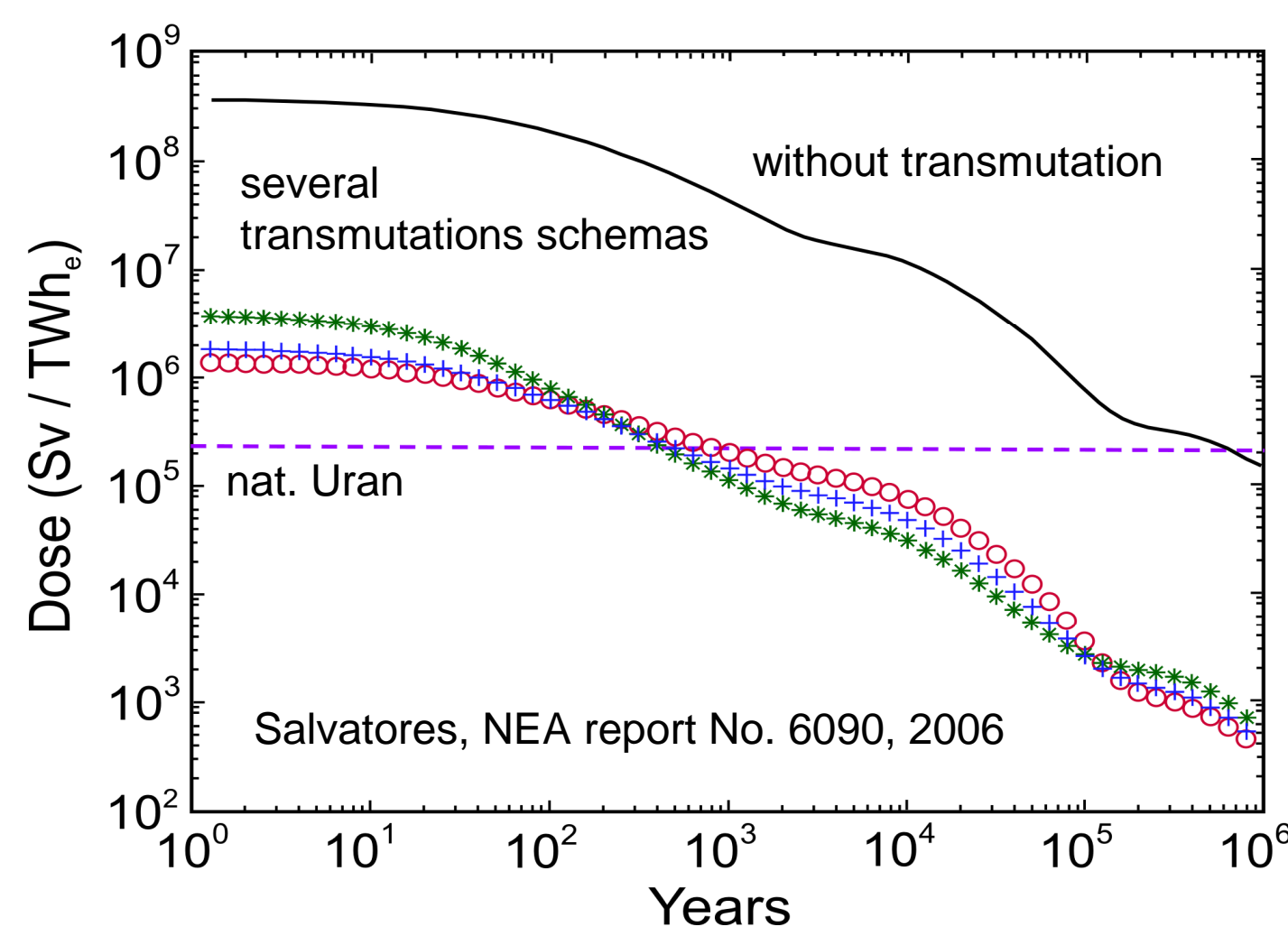
The nELBE setup for measurements of neutron-induced reactions

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Transmutation of long-lived radio-nuclides

- **Transmutation of radioactive waste = convert long-lived radio-nuclides into short-lived ones** → simplification of final disposal



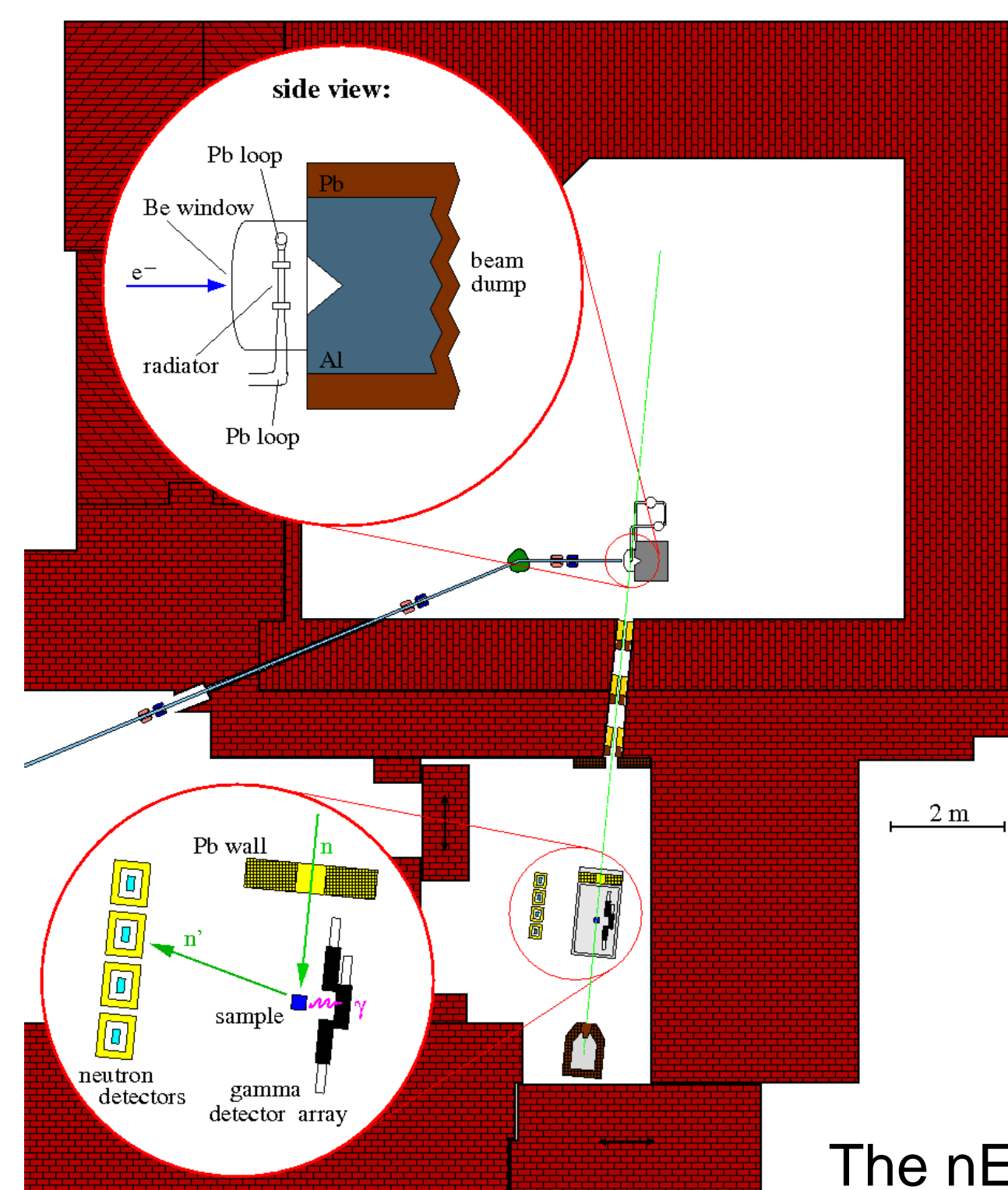
- **Without transmutation:** Radio-toxicity of untreated nuclear waste from conventional nuclear reactors reaches level of original Uranium after several **100 000 years** (—)
- **With transmutation:** Within less than **1000 years** level of original Uranium is reached (- -)

- **Fundamental processes** of inelastic neutron-scattering, neutron-capture and neutron induced fission **have to be well understood** to design transmutation facilities.

- **Fast, non moderated neutrons can both:**
 - 1.transmute** long-live fission fragments
 - 2.fission** transuranic elements: Plutonium (and other minor actinides) will be energetically used, too. Existing quantities can be burned up.

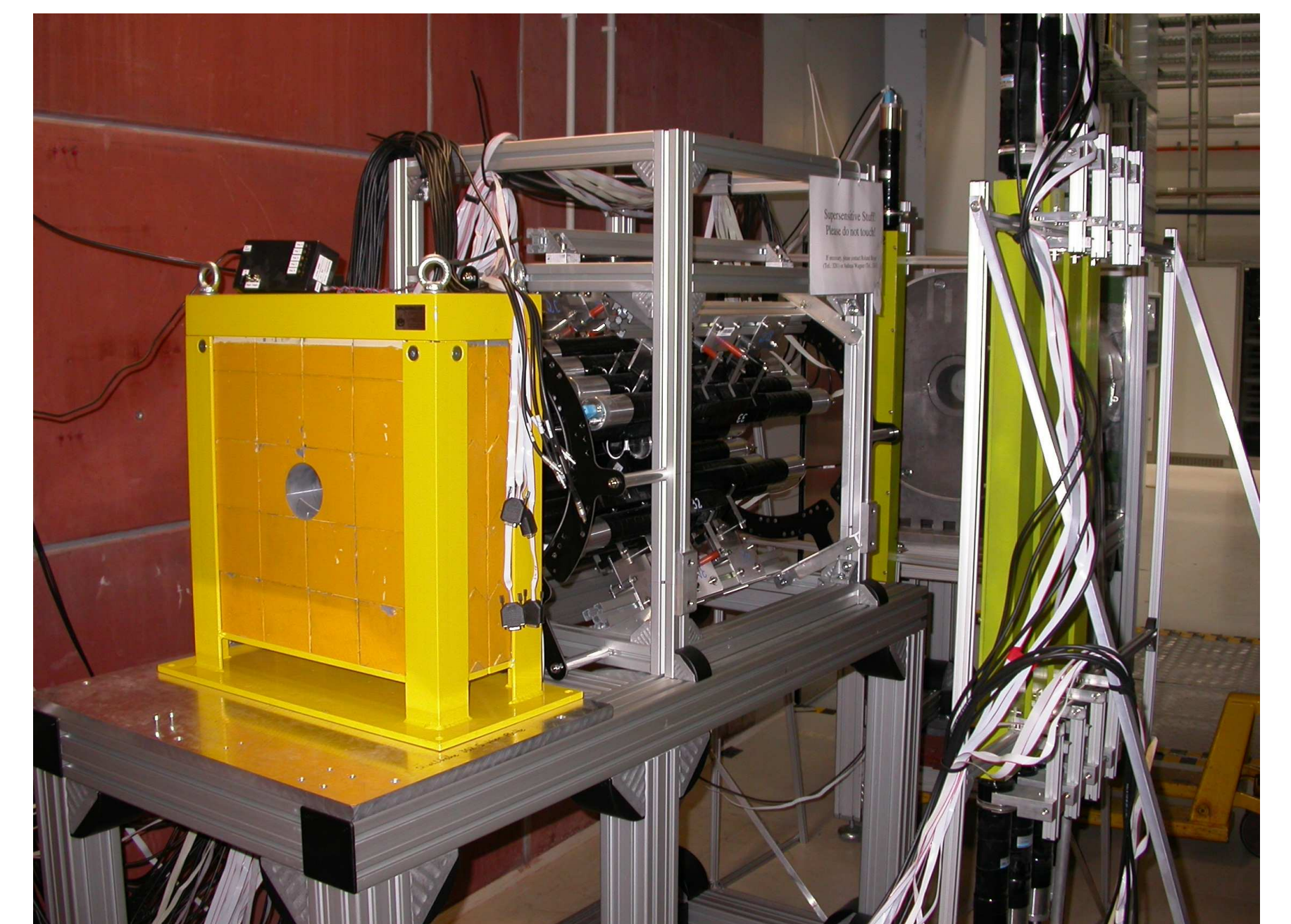
- **At nELBE reaction cross sections of neutron-induced processes will be measured to high precision.**

The nELBE neutron time-of-flight facility



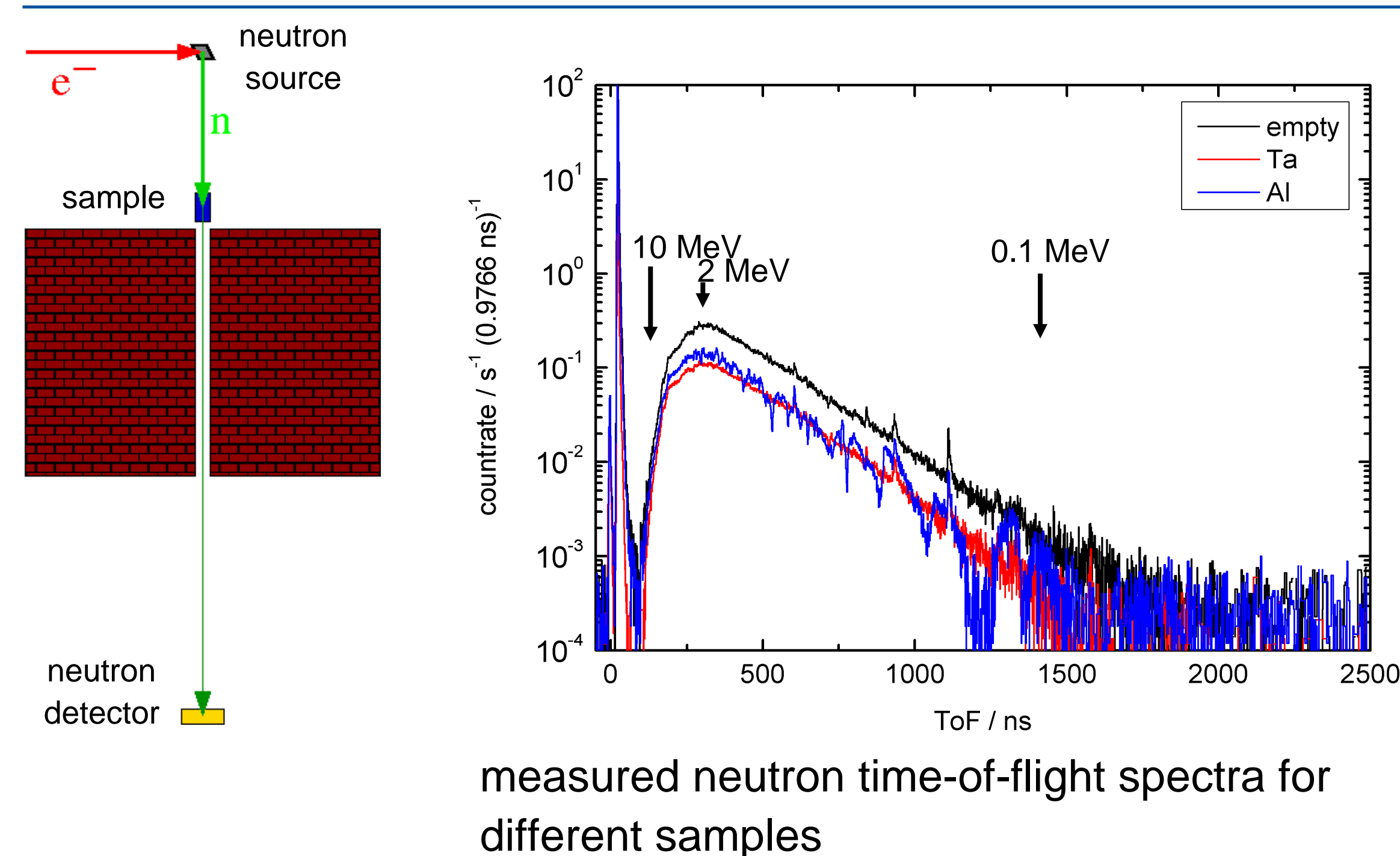
- Neutrons are produced by the ELBE electron beam impinging on a liquid lead target
- Repetition rate: 101-202 kHz
- Flight path: 5-7 m
- Neutron intensity: $1.5 \cdot 10^7 \text{ cm}^{-2} \text{ s}^{-1}$
- Neutron energy range: $100 \text{ keV} < E_n < 10 \text{ MeV}$ (energy range compare-able with fast reactors)
- Neutron energy resolution: $\Delta E/E < 1 \%$

The nELBE detector setup:
 - 5 plastic scintillation detectors to detect the neutrons
 - an array of up to 42 BaF₂ crystals for photon detection



The nELBE neutron time-of-flight facility

Transmission (n,tot)

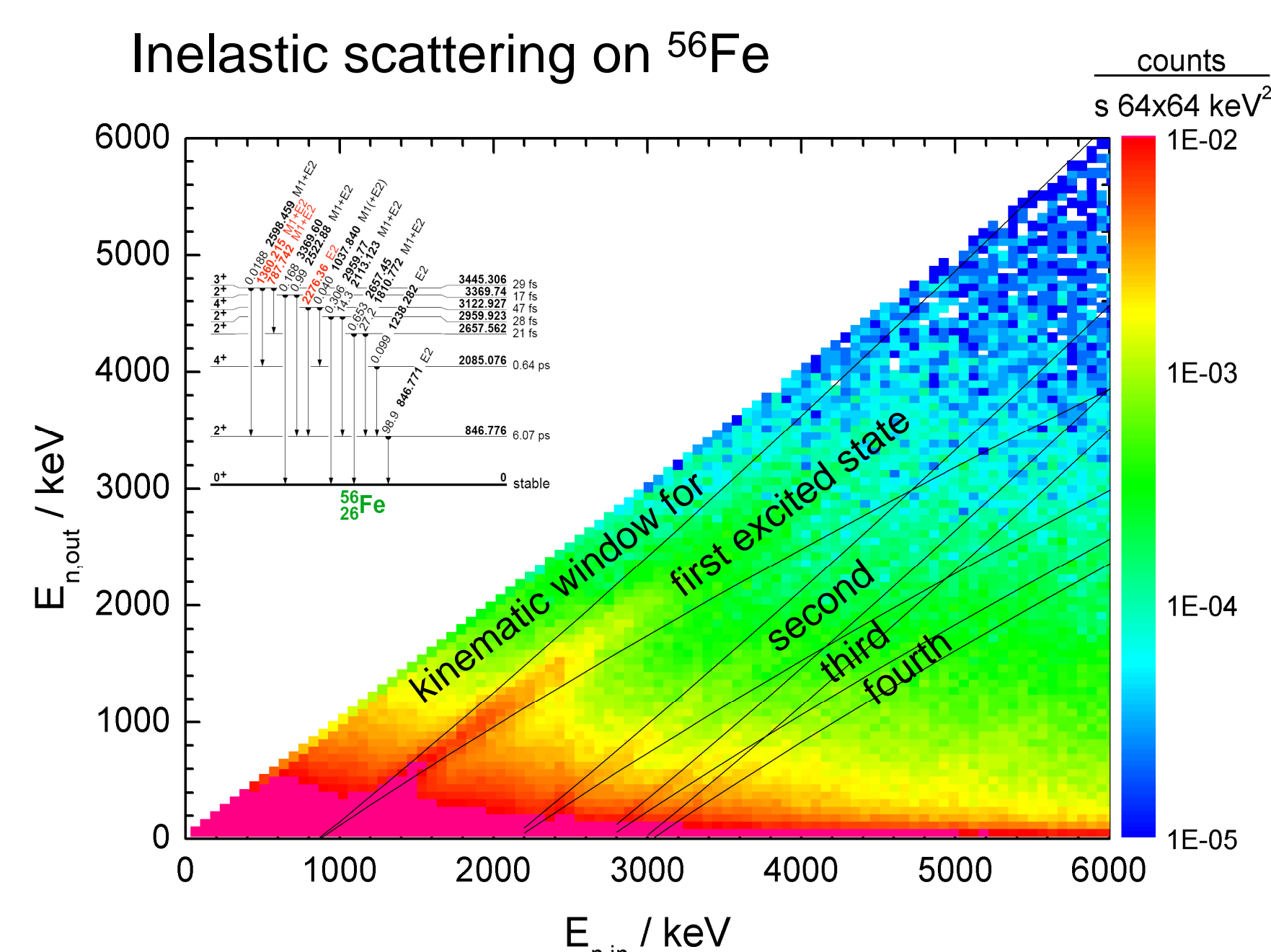
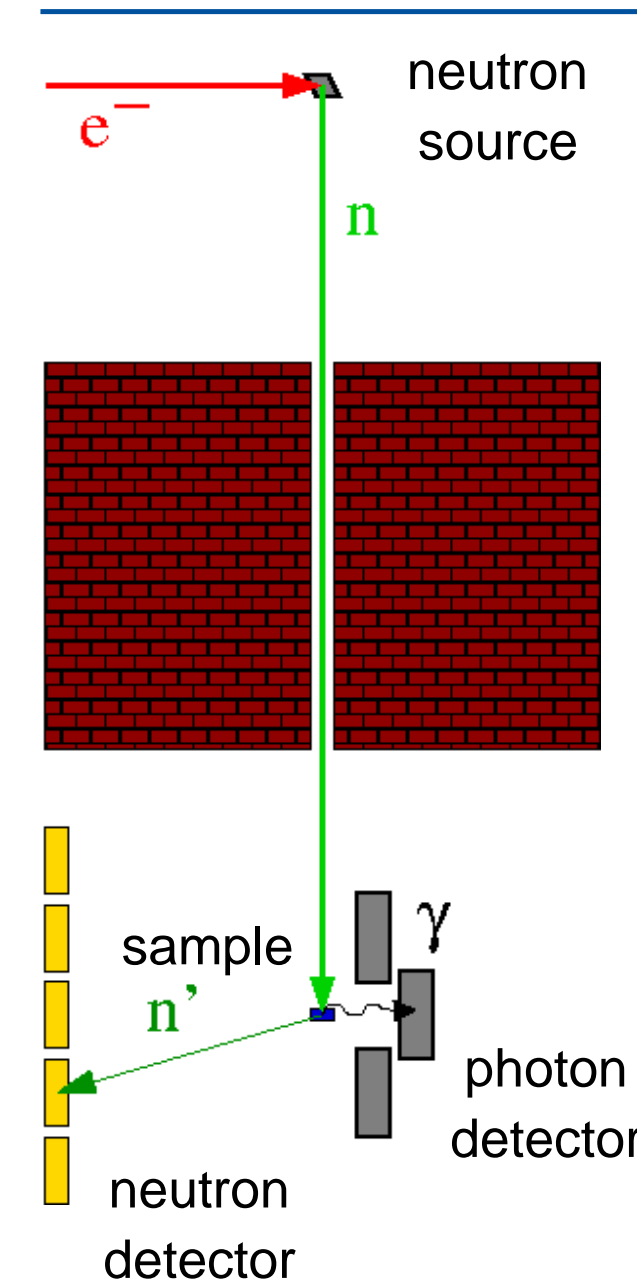


→ total neutron cross section determined by:

$$\sigma_{tot} = \frac{\ln \dot{N}_{empty} - \ln \dot{N}_{sample}}{\rho / M_{mol} \cdot N_A \cdot d}$$

ρ ... sample density
 M_{mol} ... molar mass
 d ... sample thickness

Inelastic scattering (n,n'γ)



kinematic window = allowed range of $E_{n,out}$ for given $E_{n,in}$, calculated in relativistic kinematics considering scattering angle, excitation energy and experimental uncertainties

$E_{n,in}$... energy of the incoming neutron, given by detection time of the photon ($\Delta E/E$ ca. 1 %)
 $E_{n,out}$... energy of the scattered neutron, given by time difference between photon and neutron detection ($\Delta E/E$ ca. 10 %)

→ Inelastic neutron scattering cross section determined by:

$$\sigma_{tot} = \frac{\dot{N} / \epsilon_{BaF2} / \epsilon_{Pl}}{\Phi \cdot N_{Targ}}$$

\dot{N} ... reaction detection rate
 Φ ... incoming neutron flux
 N_{Targ} ... number of nuclei in the sample



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